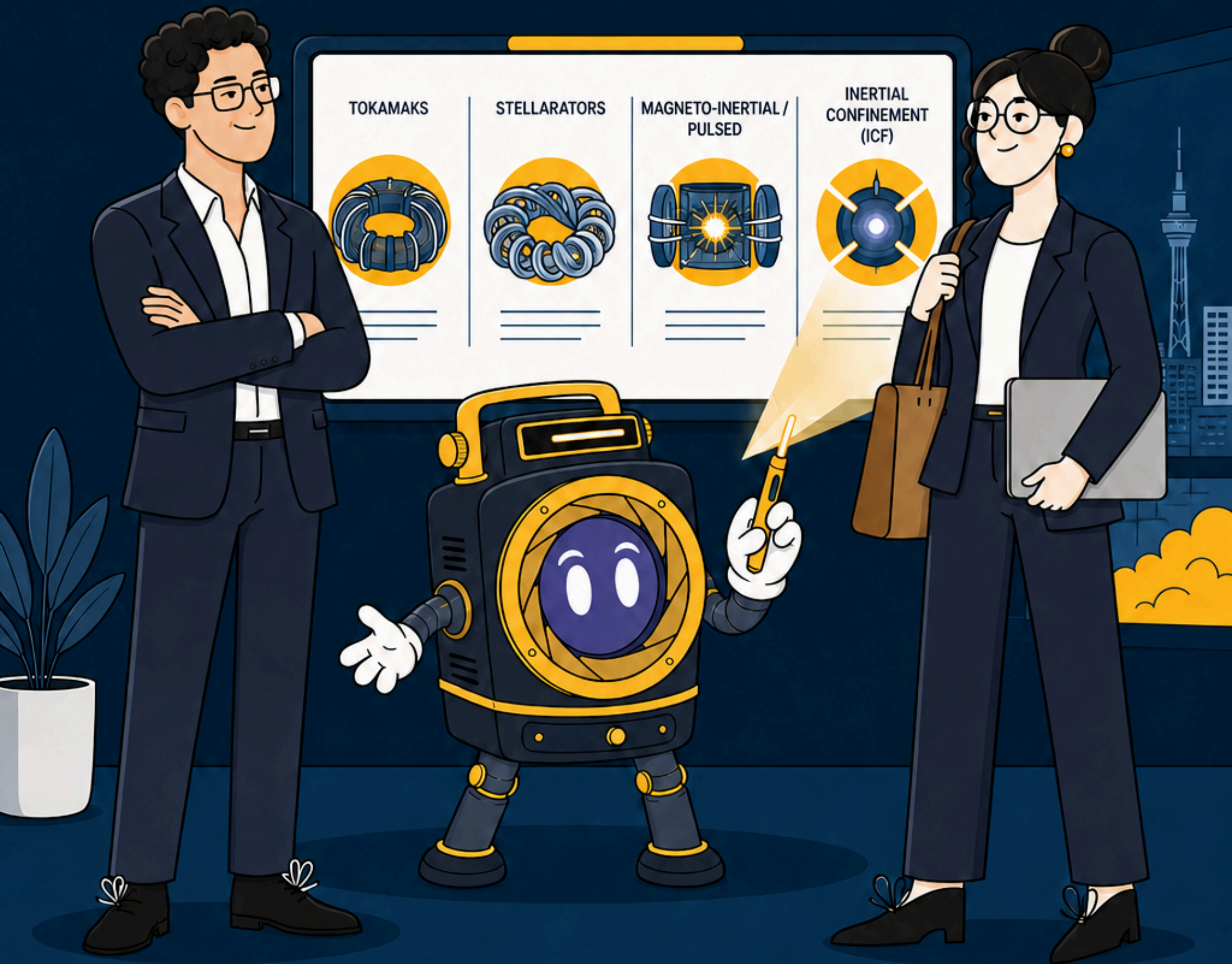


# Four Fusion Confinement Technologies:

## An Investor's Field Guide



# A Quick Primer: What Fusion Confinement Actually Is

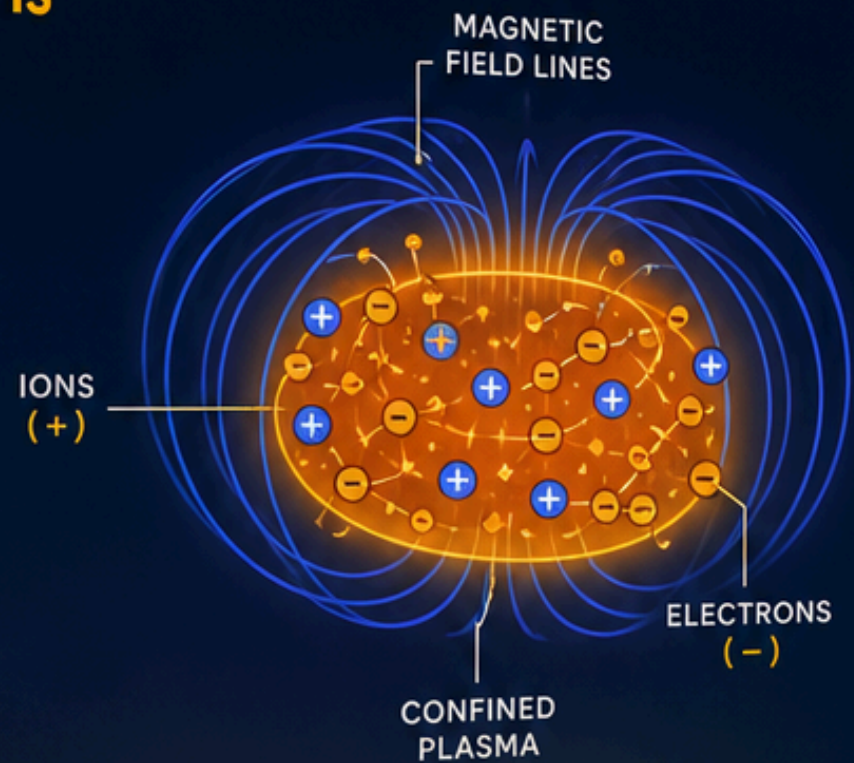
Fusion occurs when two light nuclei (typically deuterium and tritium, both isotopes of hydrogen) are forced together at extreme temperatures, around 100–150 million degrees Celsius. At those temperatures, the hydrogen fuel isn't a gas anymore. It's a **plasma**: a soup of free electrons and ions. The challenge is holding that plasma in place long enough and densely enough for fusion reactions to sustain themselves.

"Confinement" is the word physicists use for this holding problem. Different approaches solve it with different tools: magnetic fields, inertia, mechanical compression, or some combination.

The key performance metric is the **Lawson criterion** (or its modern successor, the **triple product**), which combines three variables: **plasma density** ( $n$ ), **temperature** ( $T$ ), and **energy confinement time** ( $\tau$ ). When the product  $n \times T \times \tau$  exceeds a critical threshold, the plasma becomes self-heating, or "burning," and can sustain fusion without continuous external energy input.

Every approach below is, at root, a different strategy for hitting that threshold.

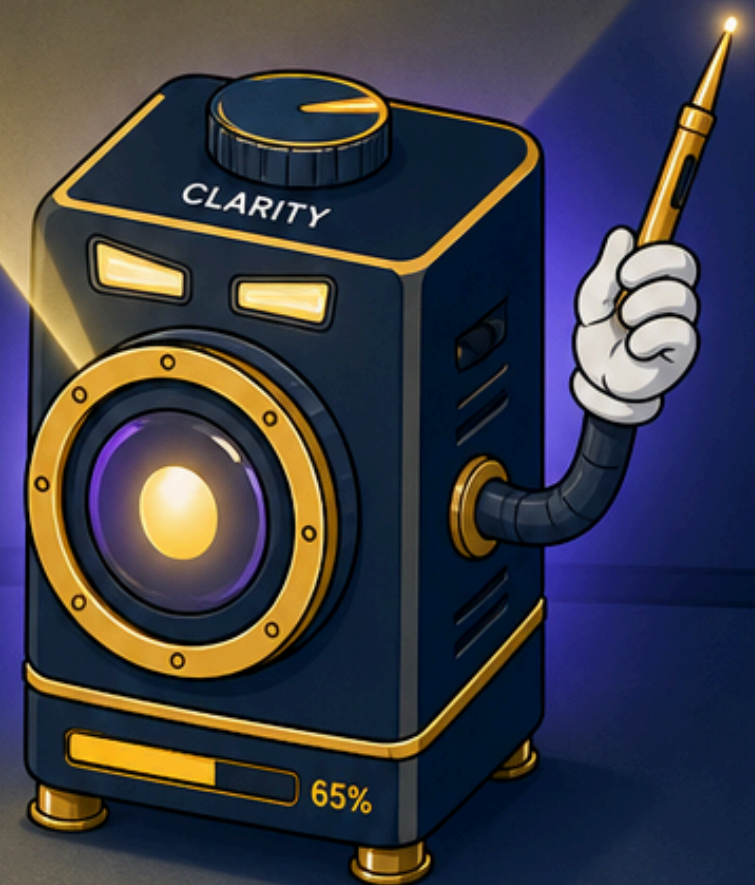
## PLASMA CONFINEMENT



## TRIPLE PRODUCT (LAWSON CRITERION)

$$n \times T \times \tau \geq (nT\tau)_c$$

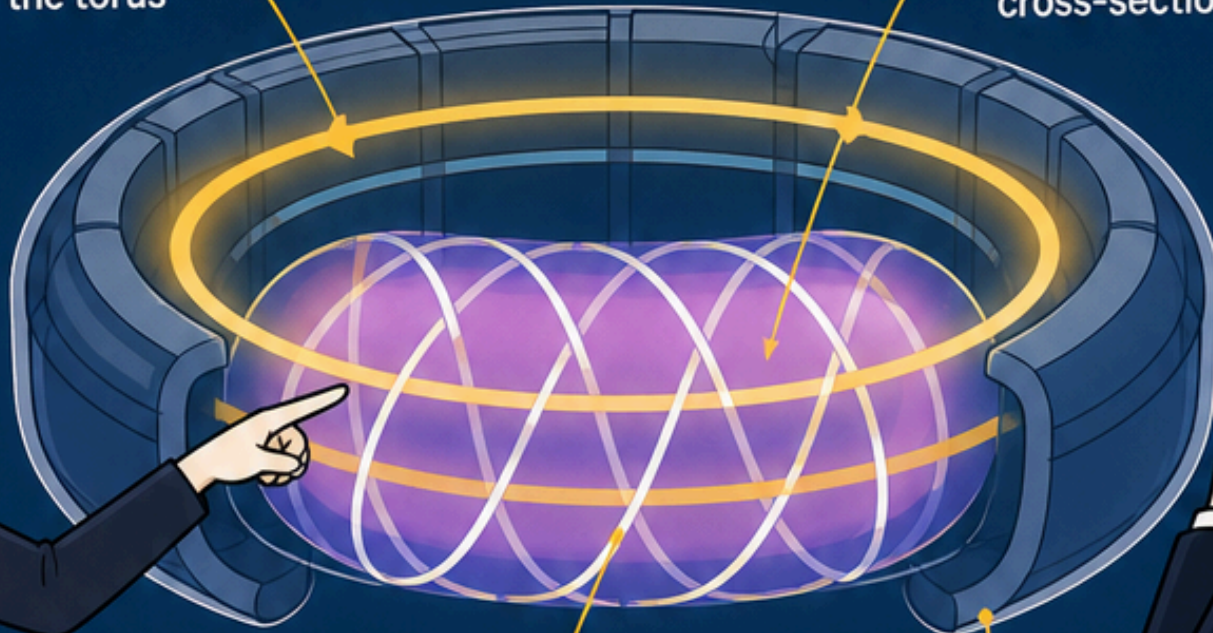
$n$ PLASMA DENSITY $(n)$	$T$ TEMPERATURE $(T)$	$\tau$ ENERGY CONFINEMENT TIME $(\tau)$	$(nT\tau)_c$ CRITICAL THRESHOLD
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# 1. Tokamaks: The Established Approach

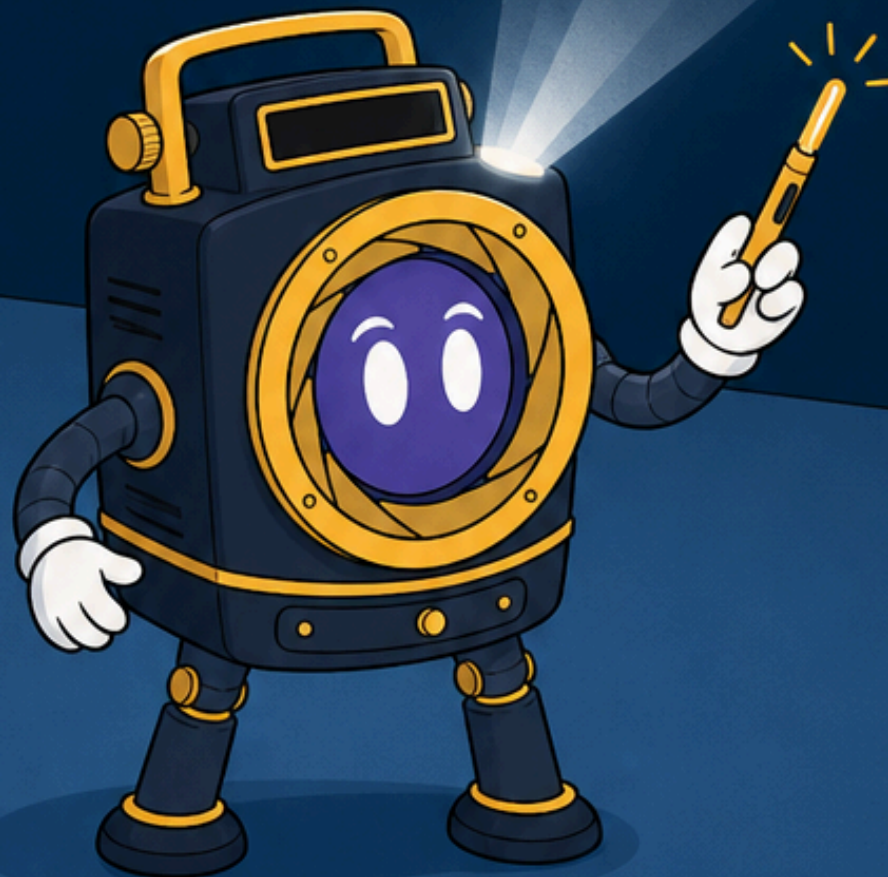
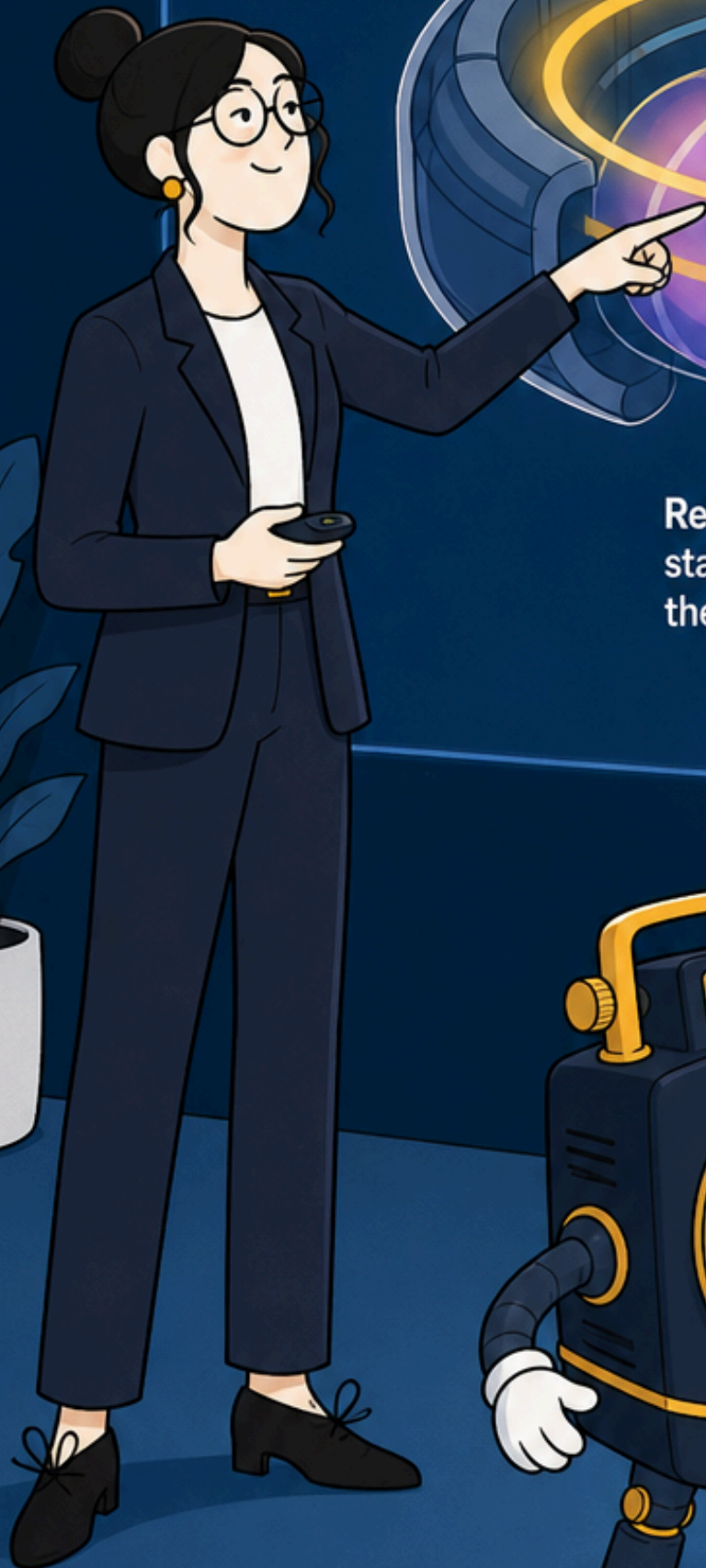
Toroidal  
Field (primary)  
around the torus

Poloidal  
Field (secondary)  
around the plasma  
cross-section

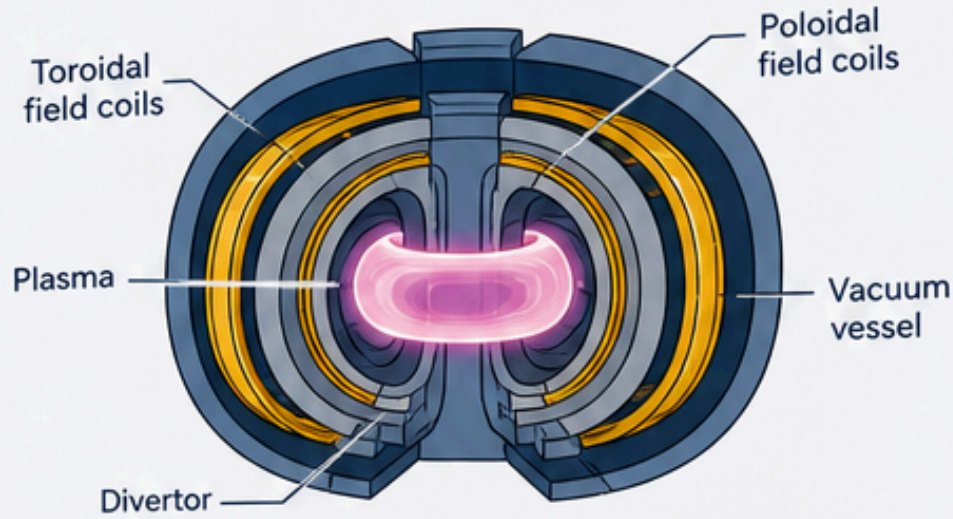


Resultant Helical Field  
stabilises and confines  
the hot plasma

Toroidal  
Vacuum Chamber



# TOKAMAKS: MATURITY AND MILESTONES



$$Q=10$$

$$\frac{500 \text{ MW}}{50 \text{ MW}}$$



## ITER milestones (updated baseline)

- First Plasma: ~2033–2034
- Full D-T operations: ~2039–2040
- Design goal: 500 MW from 50 MW heating input ( $Q = 10$ )



## CFS SPARC

- First plasma: late 2026 / early 2027
- ARC plant in Virginia (Chesterfield County)
- Backed by Google; offtake discussions underway



## Confinement time ( $\tau$ )

- Typical: a few seconds
- CEA WEST record: >1,000 s (early 2024)
- Further progress reported in 2025



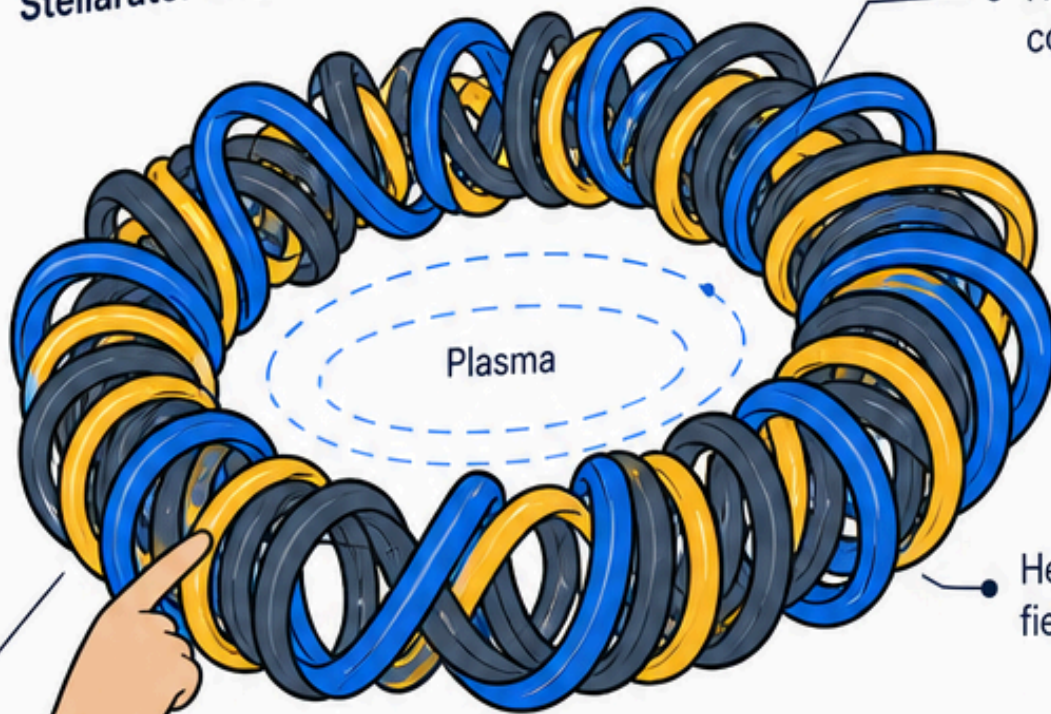
## Engineering tradeoffs

- Favors scale (ITER: 23,000 tonnes)
- High CAPEX and long timelines
- Disruptions risk
- Neutron damage to first wall/blanket
- Tritium breeding unresolved at scale



## 2. Stellarators: Cleaner Physics, Harder Engineering

Stellarator Magnetic Coils



Non-axisymmetric coil geometry

Helical magnetic field configuration

External coils  
(no plasma current)



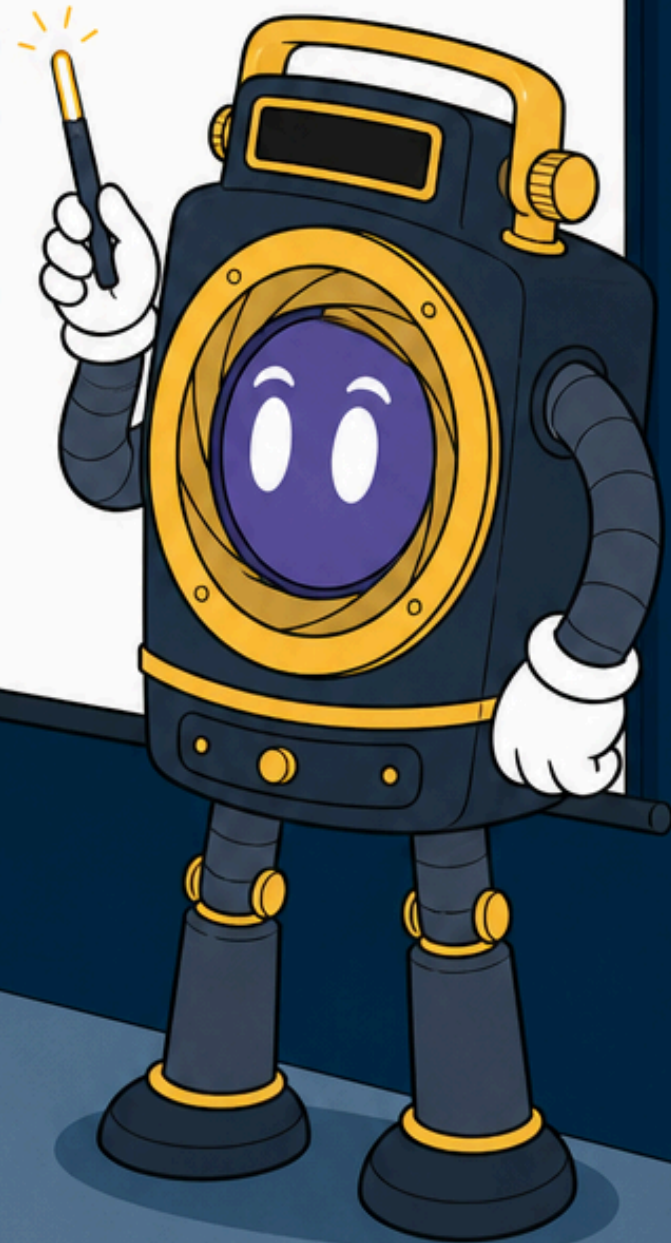
Steady-state operation



No disruptions



Continuous power  
by design



## 2. Stellarators: Maturity and milestones

Stellarators have historically lagged tokamaks on peak performance, partly because complex coil geometries made it difficult to optimise magnetic field quality. That gap has been closing.

**Wendelstein 7-X (W7-X)**, operated by the Max Planck Institute for Plasma Physics in Greifswald, Germany, is the world's largest stellarator and the primary proving ground for the approach. W7-X has reported significant progress from its OP2 experimental campaigns, with results from 2024–2025 indicating advances in both triple product and long-duration plasma performance. Investors should consult the latest Max Planck IPP publications and press releases for confirmed figures, as specific performance claims from recent campaigns are still working through peer review.

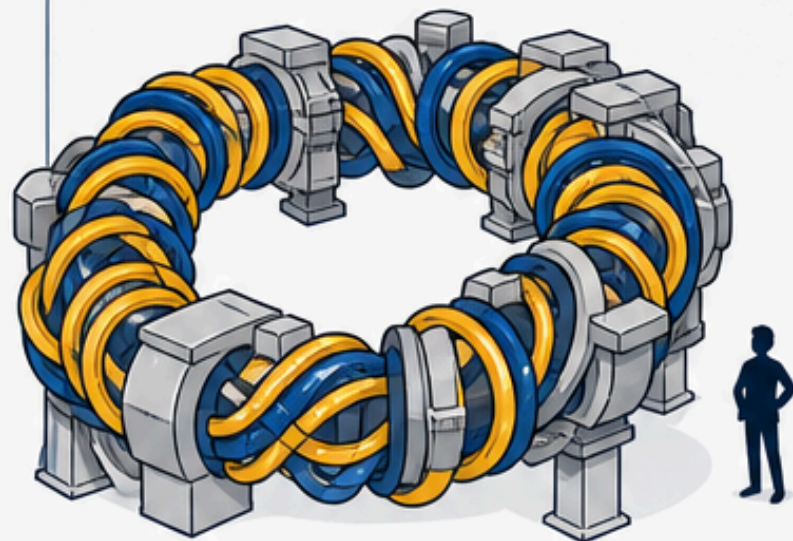
**Proxima Fusion**, a German startup spun out of Max Planck IPP's W7-X programme, has raised significant funding to advance its commercial stellarator power plant concept — investors should verify the latest confirmed round figures against Proxima's official announcements, as funding details have evolved through multiple rounds. W7-X has also become the reference point for several other private stellarator ventures globally.



### Engineering tradeoffs

The coil fabrication challenge for stellarators is not trivial. W7-X has 50 non-planar superconducting coils, each with a unique and precisely calculated three-dimensional shape. The manufacturing tolerances are measured in tenths of millimetres across metre-scale structures. Modern computational optimisation and advances in high-temperature superconductor manufacturing are reducing this barrier, but it remains the principal cost risk for stellarator commercialisation. The offset: no disruptions, no plasma-current management, and a natural pathway to continuous operation make the steady-state power economics substantially easier to model than for tokamaks.

### W7-X: COMPLEX NON-PLANAR COIL ARRANGEMENT



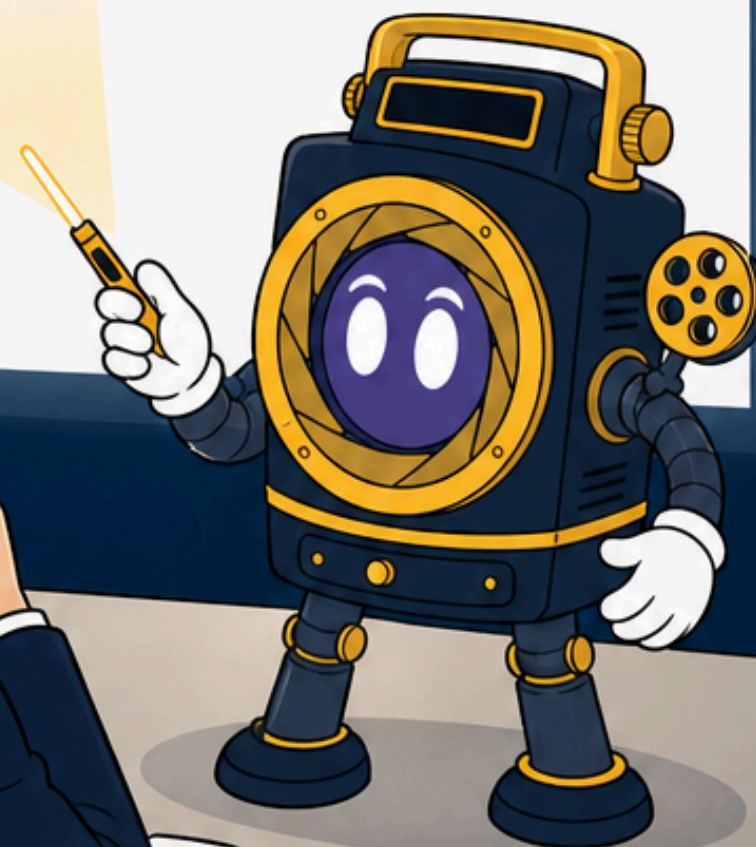
OP2 RESULTS (2024–2025):  
ADVANCES IN TRIPLE PRODUCT  
AND LONG-DURATION  
PLASMA PERFORMANCE



GLOBAL REFERENCE POINT  
FOR PRIVATE STELLARATOR  
VENTURES



COMMERCIAL PATHWAY  
BEING ADVANCED



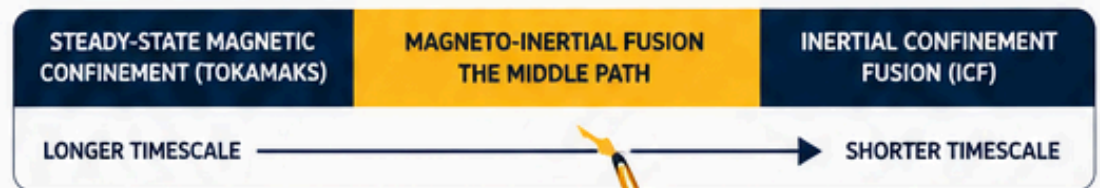
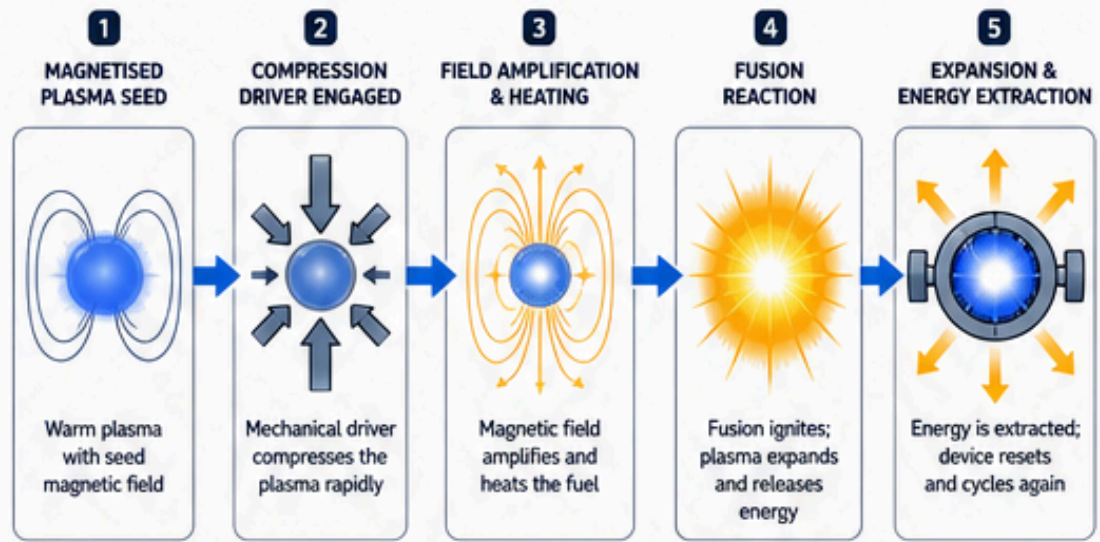
### 3. Magneto-Inertial and Pulsed Approaches: The Middle Path


#### The physics

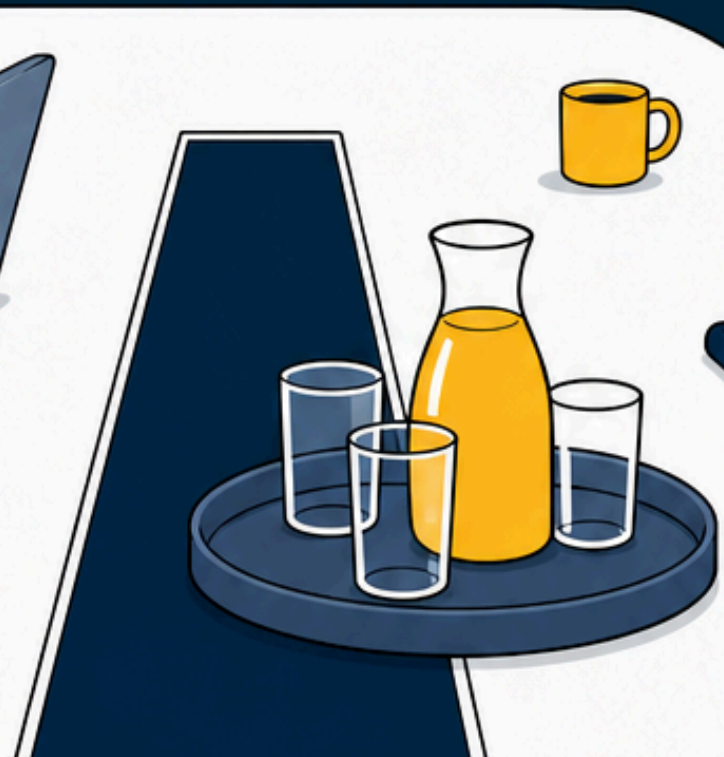
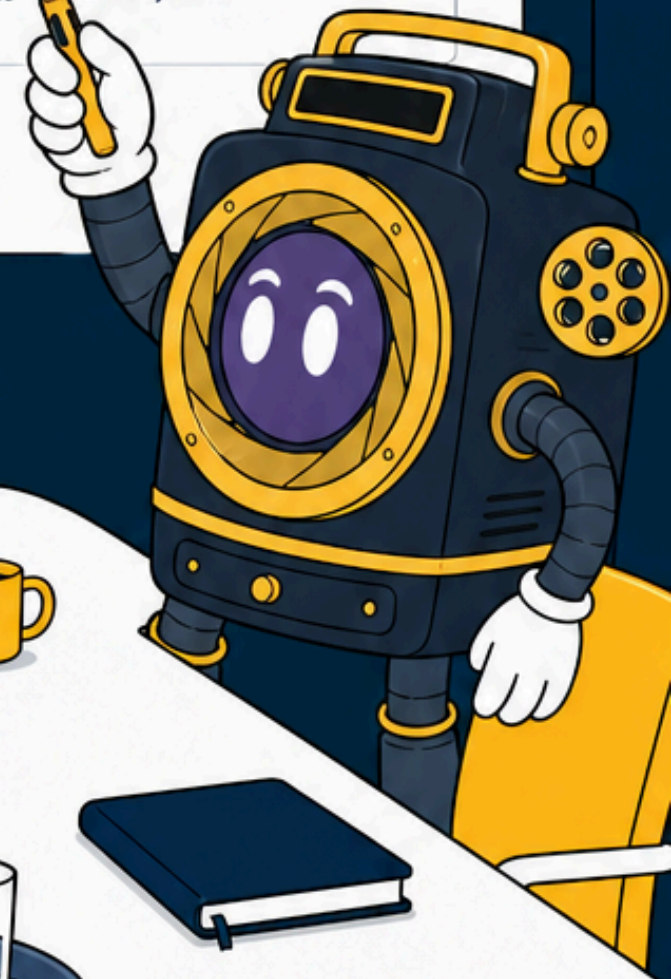
Magneto-inertial fusion (MIF), sometimes called magnetised target fusion, sits between magnetic confinement and inertial confinement. The core idea: start with a magnetised plasma that is warm but not yet at fusion temperatures. Then compress it rapidly, using a mechanical driver (spinning liquid metal, plasma jets, or magnetic coils), on a timescale fast enough that inertia keeps the plasma together during compression, but slow enough that the embedded magnetic field suppresses heat loss and assists containment.

In practical terms, a "seed" magnetic field thermally insulates the plasma before the compression phase amplifies the internal field and heats the fuel to fusion conditions. The resulting reaction causes the plasma to expand and push back against the confining field. The device then cycles again. This pulsed operation — fire, compress, extract energy, repeat — distinguishes MIF from both the steady-state magnetic approaches above and from nanosecond-pulse ICF below.

#### THE PULSED COMPRESSION CYCLE



 Pulsed operation: fire, compress, extract energy, repeat. MIF bridges the gap between steady-state magnetic systems and nanosecond-pulse ICF.



# 3. Magneto-Inertial and Pulsed Approaches: Maturity and milestones

Helion Energy has a 2023 power purchase agreement with Microsoft targeting delivery of 50 MW by 2029 — a timeline widely regarded as highly ambitious and not publicly updated as of mid-2026.

Helion has reported that its seventh-generation prototype recovered electricity from fusion plasma interactions in 2025, a claimed milestone that has not been independently verified; investors should note the important distinction between electricity recovered from the process and net electricity generation.

General Fusion, a Canadian company, is preparing to demonstrate its liquid-metal-compression MIF machine (LM26) with the potential to reach engineering breakeven.

Zap Energy's Z-pinch approach, which uses a flowing plasma compressed by its own self-generated magnetic field, has also attracted significant investment.

## GENERAL FUSION LM26

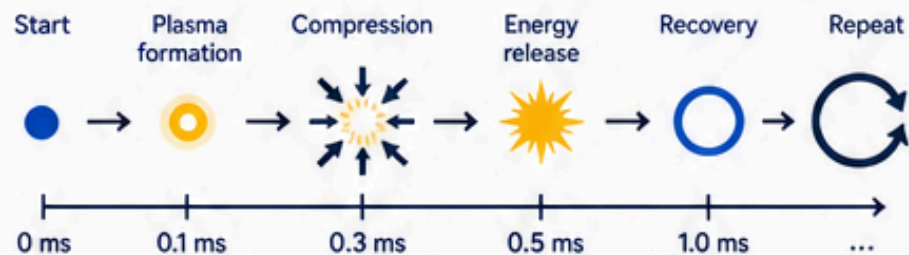
Truck-sized compression vessel vs. building-sized tokamak



← ~12 m long →

The confinement time in MIF is inherently brief per pulse — microseconds to milliseconds — but the approach aims for high repetition rates (many pulses sustained average power output).

## MIF PULSE CYCLE (EXAMPLE)



### Engineering tradeoffs



The engineering footprint for MIF devices is potentially much smaller and cheaper than large tokamaks.



The key risks are durability (high-repetition mechanical or magnetic drivers must survive millions of cycles) and driver efficiency (compressing the plasma efficiently enough to achieve net electricity at the plant level, not just scientific Q).



Direct electricity conversion, used by Helion, avoids traditional steam turbine losses but requires its own engineering development.



The pulse-and-recovery power profile also demands thoughtful power conversion engineering to deliver smooth grid output.



# 4. Inertial Confinement Fusion (ICF): The Laser Path

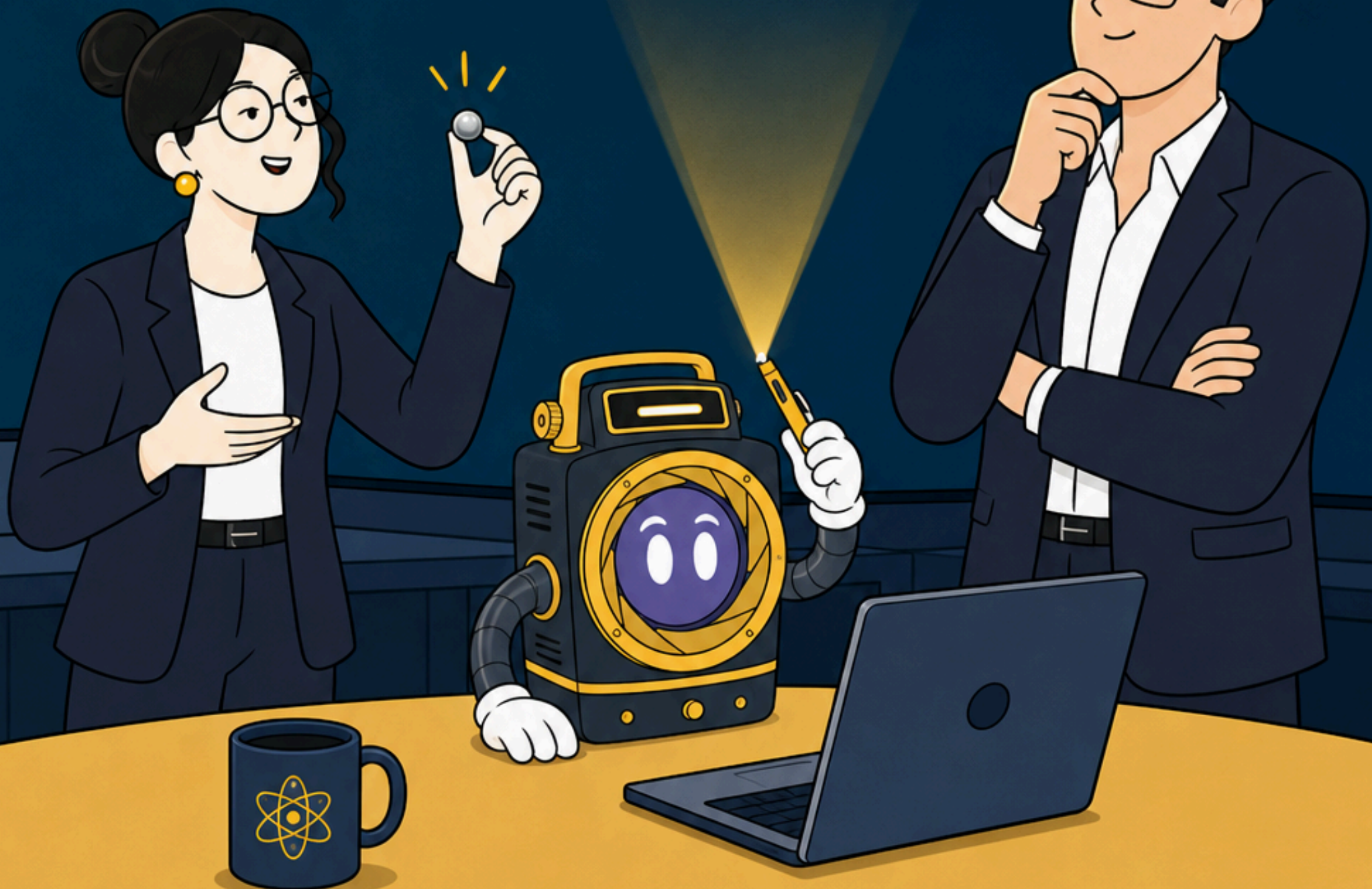
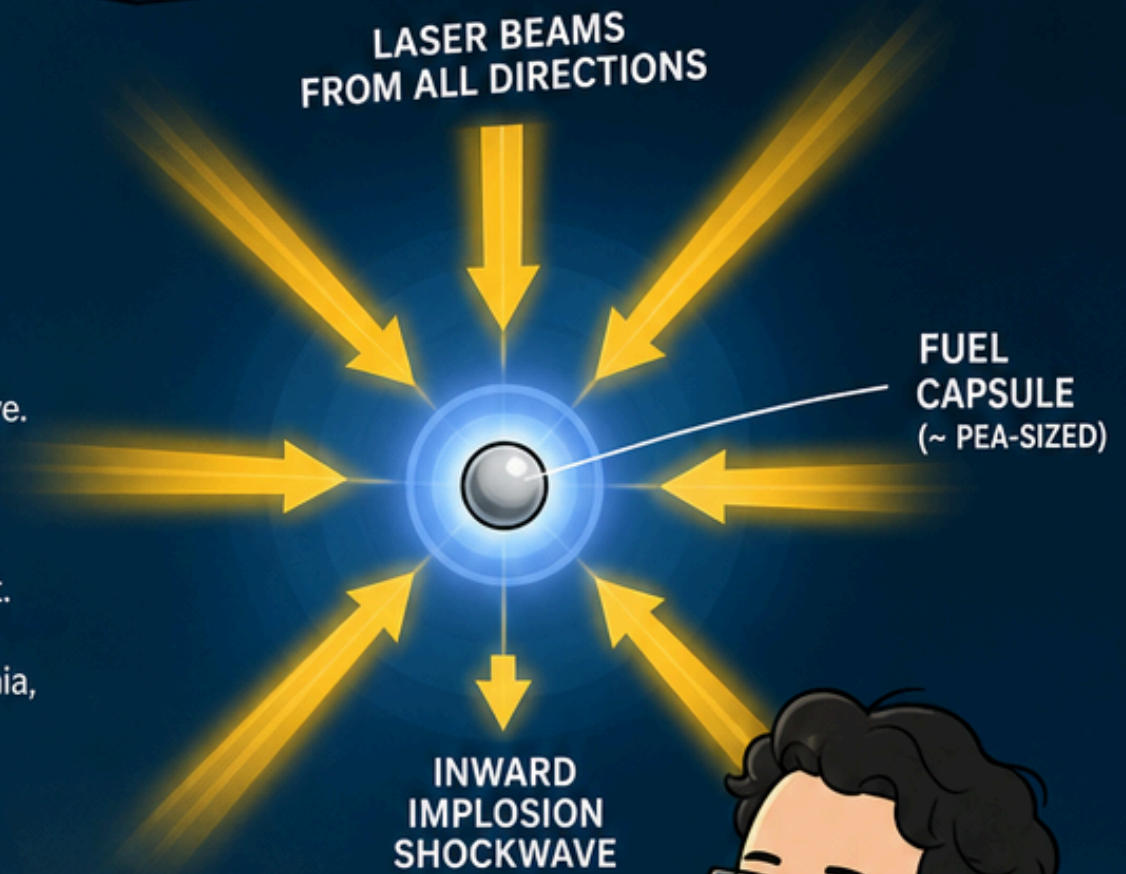
## The physics

ICF takes a completely different approach. Instead of holding plasma in a magnetic bottle, it compresses a tiny fuel capsule so fast and so hard that the plasma's own inertia keeps it together long enough for fusion to ignite.

The process takes nanoseconds. A set of laser beams or particle beams simultaneously irradiate a small sphere (roughly the size of a pea) from all directions, ablating the outer layer and driving an inward implosion shockwave.

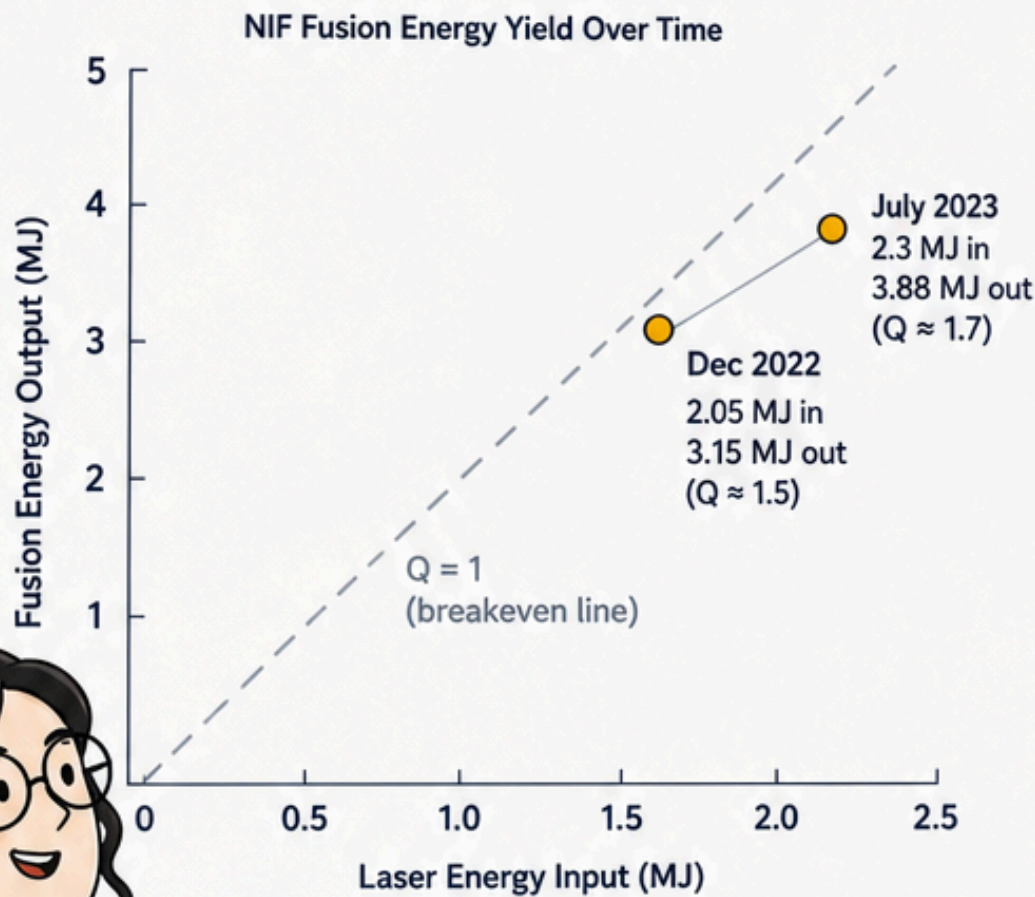
The core of the capsule reaches temperatures and densities sufficient for fusion. The compressed plasma ignites and releases energy before it has time to fly apart.

At the National Ignition Facility (NIF) in Livermore, California, the lasers heat the inside of a gold hohlraum (a small cylindrical enclosure), which converts laser energy into X-rays that then symmetrically implode the capsule. This is called indirect-drive ICF.



# 4. Inertial Confinement Fusion (ICF): Maturity and milestones

NIF achieved scientific breakeven in December 2022 ( $Q \approx 1.5$ )



Source: LLNL / NIF Official Announcements



**System Reality:**  
~300 MJ of electrical energy required to deliver a 2 MJ shot

Wall-plug efficiency  $\ll 1$  at system level



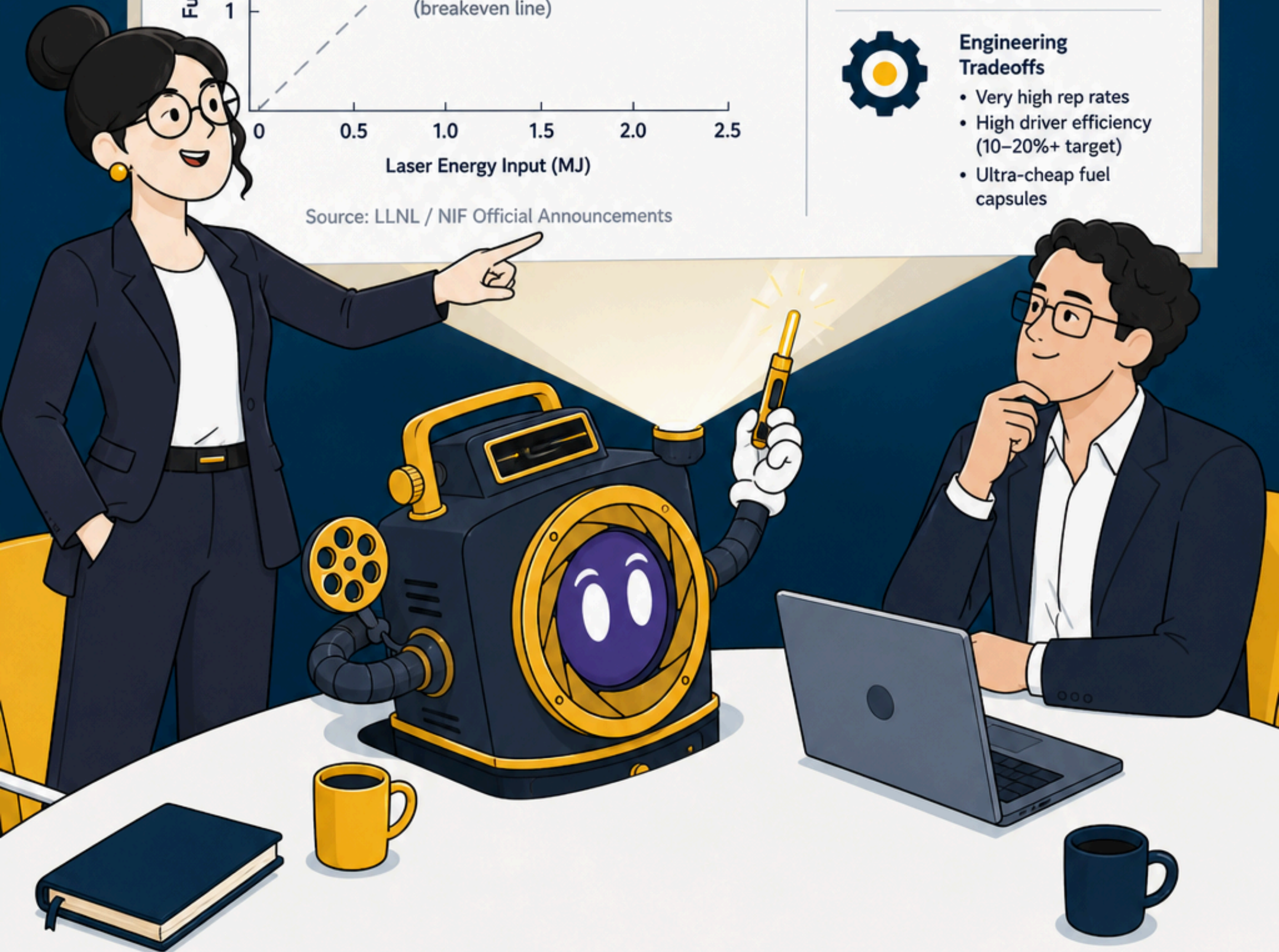
**Built for Science, Not for Power**

NIF is a weapons research facility, not a power plant prototype



**Engineering Tradeoffs**

- Very high rep rates
- High driver efficiency (10–20%+ target)
- Ultra-cheap fuel capsules



# Investor Comparison: The Five Dimensions That Matter



1 Confinement time ( $\tau$ )



2 Energy gain (Q)





3 Engineering complexity & materials durability (neutron flux)



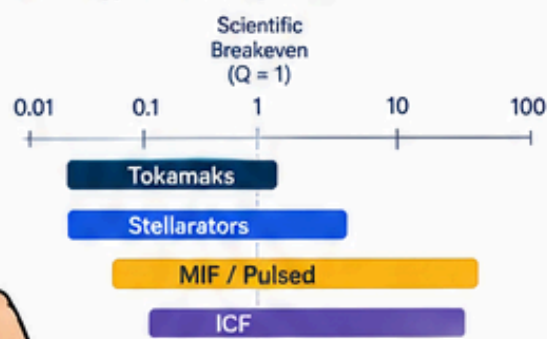
4 Pulse vs. continuous operation implications



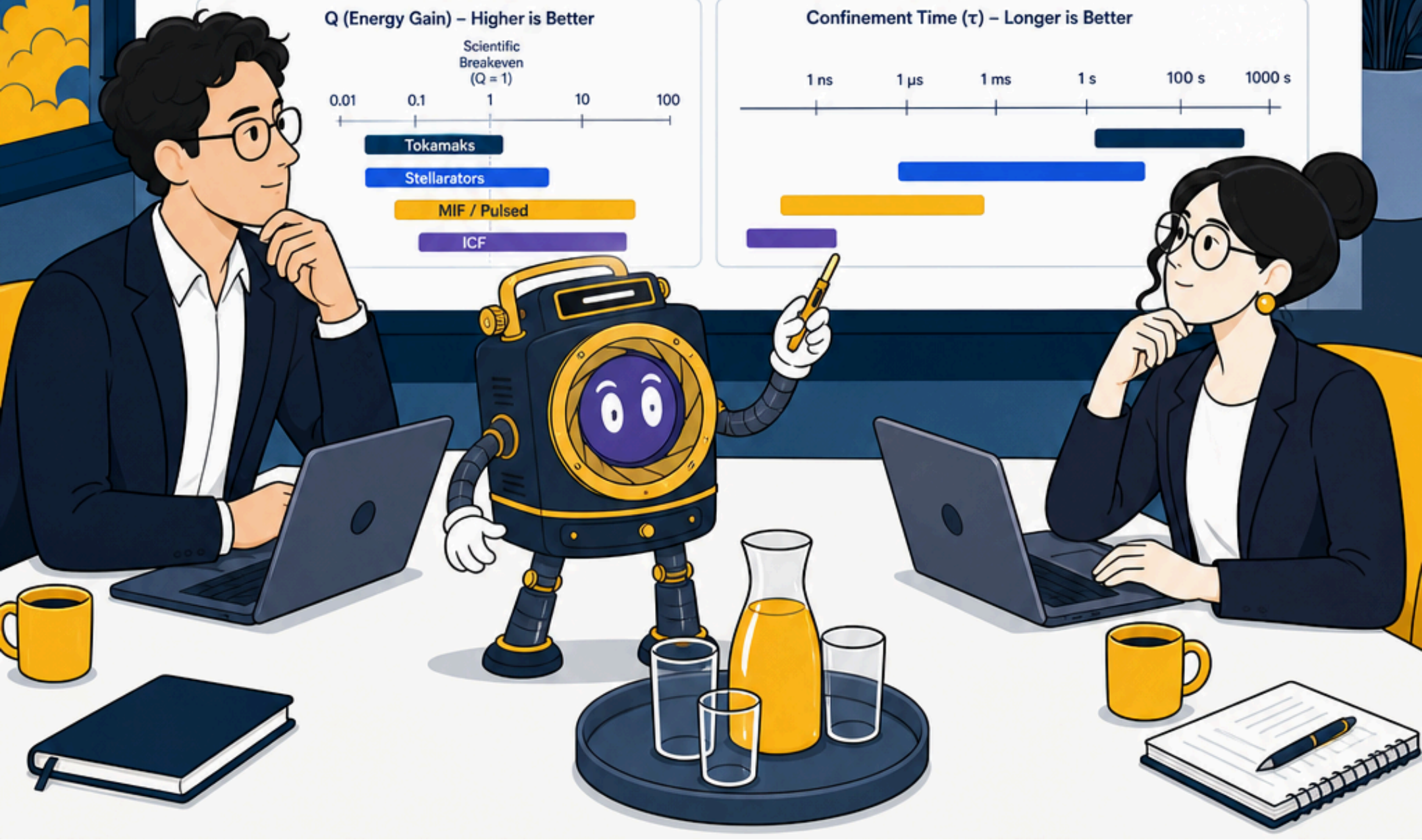
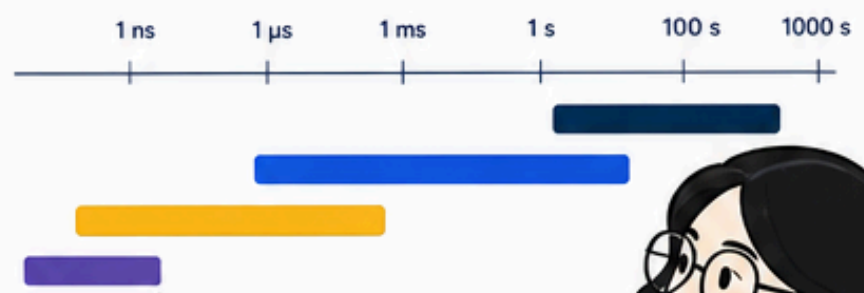
5 Realistic pilot-plant timelines

 <b>Tokamaks</b>	Seconds to minutes ITER: 400–600 s (target)	Extrapolated $Q \approx 1.1$ (JT-60U, JET ~1998)	<b>High</b> Disruption events + steady neutron bombardment	<b>Continuous</b> Compatible with steam turbines and grid	<b>Pilot plants:</b> early 2030s <b>Commercial:</b> late 2030s–2040s
 <b>Stellarators</b>	Seconds to minutes Steady-state operation	Extrapolated $Q \approx 1+$ (W7-X improving)	<b>High</b> Steady neutron exposure easier to manage	<b>Continuous</b> Compatible with steam turbines and grid	<b>Pilot plants:</b> late 2030s <b>Commercial:</b> 2040s+
 <b>MIF / Pulsed (Magneto-inertial)</b>	Microseconds to milliseconds per pulse High repetition target	Target $Q > 1$ (engineering level) Helion: targeting $> 1$ next-gen	<b>High</b> Neutron damage similar to tokamaks; pulsed mechanical stresses	<b>Pulsed</b> Requires power smoothing or direct conversion (e.g., Helion DCE)	<b>Pilot plants:</b> 2030–2035 Helion: 50 MW target by 2029 (ambitious)
 <b>ICF (Inertial Confinement Fusion)</b>	Nanoseconds per shot	Target $Q > 1$ (Dec 2022 achieved) Wall-plug $Q \ll 1$ (current)	<b>Very High</b> Chamber must survive millions of intense pulses	<b>Pulsed</b> Requires extreme repetition rate & power smoothing	<b>Pilot plants:</b> 2035–2045 Driver efficiency & rep-rate need ~100x improvement













Q (Energy Gain) – Higher is Better



Confinement Time ( $\tau$ ) – Longer is Better



# What This Means Before You Write the Cheque

APPROACH	KEY STRENGTHS	KEY RISKS	EXAMPLE / SIGNAL	COMMERCIAL OUTLOOK
 <p><b>TOKAMAKS</b></p>	<ul style="list-style-type: none"> <li>Deepest institutional knowledge base</li> <li>Most mature supply chain</li> </ul>	 <p>High cost and timeline risk ITER's repeated delays signal complexity</p>	<p><b>ITER (public)</b> Private firms building smaller tokamaks with high-temperature superconducting magnets</p>	 <p>One of the most credible near-term commercial paths, though unproven at pilot scale</p>
 <p><b>STELLARATORS</b></p>	<ul style="list-style-type: none"> <li>Cleaner physics (no disruptions)</li> <li>Natural steady-state operation</li> </ul>	 <p>Fabrication complexity of optimised coils limits commercial scalability</p>	<p><b>W7-X (operational)</b> Advances in computational design tools improving feasibility</p>	 <p>Increasingly credible to institutional investors</p>
 <p><b>MAGNETO-INERTIAL</b></p>	<ul style="list-style-type: none"> <li>Smaller device footprint</li> <li>Potentially lower capital cost</li> </ul>	 <p>Unproven durability and power conversion engineering</p>	<p><b>Helion + Microsoft deal</b> 50 MW by 2029 timeline to watch closely</p>	 <p>Commercially significant signal since NIF's 2022 ignition</p>
 <p><b>ICF</b></p>	<ul style="list-style-type: none"> <li>Scientific gains at NIF are real and improving</li> </ul>	 <p>Large wall-plug efficiency gap Laser physics advances needed (not yet at commercial scale)</p>	<p><b>NIF (ignition)</b> Companies exploring new driver architectures</p>	 <p>Early-stage deep tech, not fusion-adjacent cleantech</p>



All four approaches are worth tracking. None are guaranteed.

The most useful question to ask any fusion company is not "what's your Q?" but "what has to be true about your power conversion, materials durability, and driver efficiency for your pilot plant to generate dispatchable grid electricity at a competitive cost — and which of those has been demonstrated?"

**That question will tell you more than the plasma physics slide will.**



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